ELSEVIER

Journal of Nuclear Materials 238 (1996) 285–286

journal of nuclear materials

Author index

Arita, Y., see Yokoi, H. 238 (1996) 163

Baldo, P.M., see Iwase, A. 238 (1996) 224
Banerjee, A.A., see Venugopal, V. 238 (1996) 218
Baraj, E., see Giménez, J. 238 (1996) 64
Bates, J.K., see Wronkiewicz, D.J. 238 (1996) 78
Bottomley, P.D.W., D.H. Wegen and M. Coquerelle , Construction of an electrode for irradiated UO_2 fuel and preliminary results concerning its use in aqueous solutions 238 (1996) 23
Bouffioux, P., see Delobelle, P. 238 (1996) 135
Bruno, J., E. Cera, L. Duro, T.E. Eriksen and L.O. Werme , A kinetic model for the stability of spent fuel matrix under oxic conditions 238 (1996) 110
Buck, E.C., see Wronkiewicz, D.J. 238 (1996) 78
Burkart, A.L., see Semino, C.J. 238 (1996) 198

Casas, I., see Giménez, J. 238 (1996) 64
Cassibba, R., see Semino, C.J. 238 (1996) 198
Cera, E., see Bruno, J. 238 (1996) 110
Christensen, H. and S. Sunder , An evaluation of water layer thickness effective in the oxidation of UO_2 fuel due to radiolysis of water 238 (1996) 70
Closs, K.D., see Kim, J.I. 238 (1996) 1
Coquerelle, M., see Bottomley, P.D.W. 238 (1996) 23

Delobelle, P., P. Robinet, P. Geyer and P. Bouffioux , A model to describe the anisotropic viscoplastic behaviour of Zircaloy-4 tubes 238 (1996) 135
De Pablo, J., see Giménez, J. 238 (1996) 64
Diaz Arocas, P. , Precipitation kinetics of U(VI) phases formed in high concentrated saline solutions 238 (1996) 96
Duro, L., see Bruno, J. 238 (1996) 110

Edwards, D.J., see Singh, B.N. 238 (1996) 244
Engelhardt, J., F. Feldmaier, M. Laske and G. Marx , Impedance measurements on UO_{2+x} in saturated NaCl solutions 238 (1996) 104
Eriksen, T.E., see Bruno, J. 238 (1996) 110
Evans, J.H. , The role of directed bubble diffusion to grain boundaries in post-irradiation fission gas release from UO_2: a quantitative assessment 238 (1996) 175
Evens, D., see Lewis, B.J. 238 (1996) 183
Ewing, R.C., see Janeczek, J. 238 (1996) 121

Faude, D., see Kim, J.I. 238 (1996) 1
Feldmaier, F., see Engelhardt, J. 238 (1996) 104
Funk, L., see Iwase, A. 238 (1996) 224

García, M.E., see Semino, C.J. 238 (1996) 198
Geckeis, H., see Loida, A. 238 (1996) 11
Geckeis, H., see Quiñones, J. 238 (1996) 38
Geyer, P., see Delobelle, P. 238 (1996) 135
Giménez, J., E. Baraj, M.E. Torrero, I. Casas and J. de Pablo , Effect of H_2O_2, NaClO and Fe on the dissolution of unirradiated UO_2 in NaCl 5 mol kg^{-1}. Comparison with spent fuel dissolution experiments 238 (1996) 64
Gnanamoorthy, J.B., see Vinoy, T.V. 238 (1996) 278
Gompper, K., see Kim, J.I. 238 (1996) 1
Grambow, B., see Loida, A. 238 (1996) 11
Grambow, B., see Quiñones, J. 238 (1996) 38

Harada, Y. , Sintering behaviour of niobia-doped large grain UO_2 pellet 238 (1996) 237
Hong, H.-S., S.-J. Kim and K.-S. Lee , Effects of alloying elements on the tensile properties and oxidation behavior of modified Zircaloy-4 in 360°C water 238 (1996) 211
Hood, G.M., see Matsuura, N. 238 (1996) 260

Iglesias, F.C., see Lewis, B.J. 238 (1996) 183
Iwase, A., L.E. Rehn, P.M. Baldo and L. Funk , Annihilation of freely migrating defects by cascade remnants in Cu–1% Au alloys 238 (1996) 224

Janeczek, J., R.C. Ewing, V.M. Oversby and L.O. Werme , Uraninite and UO_2 in spent nuclear fuel: a comparison 238 (1996) 121
Jung, P , Compositional variation of hydrogen permeability in ferritic alloys and steels 238 (1996) 189

Kessler, G., see Kim, J.I. 238 (1996) 1
Khatak, H.S., see Vinoy, T.V. 238 (1996) 278
Kim, J.I., K. Gompper, K.D. Closs, G. Kessler and D. Faude , German approaches to closing the nuclear fuel cycle and final disposal of HLW 238 (1996) 1
Kim, S.-J., see Hong, H.-S. 238 (1996) 211
Kobayashi, K., see Yokoi, H. 238 (1996) 163
Kobylyansky, G.P., see Nikulina, A.V. 238 (1996) 205
Kulkarni, S.G., see Venugopal, V. 238 (1996) 218

Laske, M., see Engelhardt, J. 238 (1996) 104
Lee, K.-S., see Hong, H.-S. 238 (1996) 211
Lewis, B.J., D. Evens, F.C. Iglesias and Y. Liu , Modelling of short-lived fission product release behaviour during annealing conditions 238 (1996) 183
Liu, Y., see Lewis, B.J. 238 (1996) 183
Loida, A., B. Grambow and H. Geckeis , Anoxic corrosion of various high burnup spent fuel samples 238 (1996) 11
Loida, A., see Quiñones, J. 238 (1996) 38

Markelov, V.A., see Nikulina, A.V. 238 (1996) 205
Marx, G., see Engelhardt, J. 238 (1996) 104
Matsui, T., see Yokoi, H. 238 (1996) 163
Matsuura, N., G.M. Hood and H. Zou , The correspondence between self-diffusion properties and melting temperatures for α-Zr and α-Ti 238 (1996) 260
Matzke, Hj., see Rondinella, V.V. 238 (1996) 44
Matzke, Hj. , Analysis of the structure of layers on UO_2 leached in H_2O 238 (1996) 58

Nakatsuka, M., see Torimaru, T. 238 (1996) 169
Nikulina, A.V., V.A. Markelov, M.M. Peregud, V.N. Voevodin, V.L. Panchenko and G.P. Kobylyansky , Irradiation-induced microstructural changes in Zr–1% Sn–1% Nb–0.4% Fe 238 (1996) 205

Ohno, H., see Yokoi, H. 238 (1996) 163
Oversby, V.M., see Janeczek, J. 238 (1996) 121

Panchenko, V.L., see Nikulina, A.V. 238 (1996) 205
Peregud, M.M., see Nikulina, A.V. 238 (1996) 205

Quiñones, J., B. Grambow, A. Loida and H. Geckeis , Coprecipitation phenomena during spent fuel dissolution. Part 1: Experimental procedure and initial results on trivalent ion behaviour 238 (1996) 38

Rama Rao, G.A., see Venugopal, V. 238 (1996) 218
Rehn, L.E., see Iwase, A. 238 (1996) 224
Robinet, P., see Delobelle, P. 238 (1996) 135
Rondinella, V.V. and Hj. Matzke , Leaching of SIMFUEL in simulated granitic water: comparison to results in demineralized water 238 (1996) 44
Roy, K.N., see Venugopal, V. 238 (1996) 218

Semino, C.J., A.L. Burkart, M.E. García and R. Cassibba , Lead/carbon steel galvanic corrosion evaluation 238 (1996) 198
Shaikh, H., see Vinoy, T.V. 238 (1996) 278
Singh, B.N., D.J. Edwards and P. Toft , Effects of neutron irradiation on mechanical properties and microstructures of dispersion and precipitation hardened copper alloys 238 (1996) 244
Sivaibharasi, N., see Vinoy, T.V. 238 (1996) 278
Sood, D.D., see Venugopal, V. 238 (1996) 218
Stroes-Gascoyne, S. , Measurements of instant-release source terms for ^{137}Cs, ^{90}Sr, ^{99}Tc, ^{129}I and ^{14}C in used CANDU fuels 238 (1996) 264
Sunder, S., see Christensen, H. 238 (1996) 70

Toft, P., see Singh, B.N. 238 (1996) 244
Torimaru, T., T. Yasuda and M. Nakatsuka , Changes in mechanical properties of irradiated Zircaloy-2 fuel cladding due to short term annealing 238 (1996) 169
Torrero, M.E., see Giménez, J. 238 (1996) 64

Venugopal, V., S.G. Kulkarni, A.A. Banerjee, G.A. Rama Rao, K.N. Roy and D.D. Sood , Determination of partial pressures of Te, Te_2, Ag, Ag_2, AgTe and Ag_2Te gaseous species over $Ag(s) + Ag_2Te(s)$ system 238 (1996) 218
Vinoy, T.V., H. Shaikh, H.S. Khatak, N. Sivaibharasi and J.B. Gnanamoorthy , Stress corrosion crack growth studies on AISI type 316 stainless steel in boiling acidified sodium chloride solution 238 (1996) 278
Voevodin, V.N., see Nikulina, A.V. 238 (1996) 205

Wegen, D.H., see Bottomley, P.D.W. 238 (1996) 23
Werme, L.O., see Bruno, J. 238 (1996) 110
Werme, L.O., see Janeczek, J. 238 (1996) 121
Wolf, S.F., see Wronkiewicz, D.J. 238 (1996) 78
Wronkiewicz, D.J., J.K. Bates, S.F. Wolf and E.C. Buck , Ten-year results from unsaturated drip tests with UO_2 at 90°C: implications for the corrosion of spent nuclear fuel 238 (1996) 78

Yasuda, T., see Torimaru, T. 238 (1996) 169
Yokoi, H., Y. Arita, T. Matsui, H. Ohno and K. Kobayashi , EXAFS study of $(La_{1-x}M_x)_2Zr_2O_7$ (M = Nd and Ce) 238 (1996) 163

Zou, H., see Matsuura, N. 238 (1996) 260

ELSEVIER

Journal of Nuclear Materials 238 (1996) 287–289

journal of nuclear materials

Subject index

Actinides *(excludes Plutonium, Thorium and Uranium)*

Coprecipitation phenomena during spent fuel dissolution. Part 1: Experimental procedure and initial results on trivalent ion behaviour, J. Quiñones, B. Grambow, A. Loida and H. Geckeis 238 (1996) 38

Impedance measurements on UO_{2+x} in saturated NaCl solutions, J. Engelhardt, F. Feldmaier, M. Laske and G. Marx 238 (1996) 104

EXAFS study of $(La_{1-x}M_x)_2Zr_2O_7$ (M = Nd and Ce), H. Yokoi, Y. Arita, T. Matsui, H. Ohno and K. Kobayashi 238 (1996) 163

Analytical Instruments and Methods

Determination of partial pressures of Te, Te_2, Ag, Ag_2, AgTe and Ag_2Te gaseous species over Ag(s) + Ag_2Te(s) system, V. Venugopal, S.G. Kulkarni, A.A. Banerjee, G.A. Rama Rao, K.N. Roy and D.D. Sood 238 (1996) 218

Ceramics *(not listed elsewhere)*

EXAFS study of $(La_{1-x}M_x)_2Zr_2O_7$ (M = Nd and Ce), H. Yokoi, Y. Arita, T. Matsui, H. Ohno and K. Kobayashi 238 (1996) 163

Chemical Reactions *(includes Electrochemical and Thermochemical Reactions)*

Construction of an electrode for irradiated UO_2 fuel and preliminary results concerning its use in aqueous solutions, P.D.W. Bottomley, D.H. Wegen and M. Coquerelle 238 (1996) 23

Coprecipitation phenomena during spent fuel dissolution. Part 1: Experimental procedure and initial results on trivalent ion behaviour, J. Quiñones, B. Grambow, A. Loida and H. Geckeis 238 (1996) 38

Analysis of the structure of layers on UO_2 leached in H_2O, Hj. Matzke 238 (1996) 58

Effect of H_2O_2, NaClO and Fe on the dissolution of unirradiated UO_2 in NaCl 5 mol kg^{-1}. Comparison with spent fuel dissolution experiments, J. Giménez, E. Baraj, M.E. Torrero, I. Casas and J. de Pablo 238 (1996) 64

An evaluation of water layer thickness effective in the oxidation of UO_2 fuel due to radiolysis of water, H. Christensen and S. Sunder 238 (1996) 70

Effects of alloying elements on the tensile properties and oxidation behavior of modified Zircaloy-4 in 360°C water, H.-S. Hong, S.-J. Kim and K.-S. Lee 238 (1996) 211

Cladding Materials

Lead/carbon steel galvanic corrosion evaluation, C.J. Semino, A.L. Burkart, M.E. García and R. Cassibba 238 (1996) 198

Compatibility and Corrosion *(includes Stress Corrosion Cracking)*

Anoxic corrosion of various high burnup spent fuel samples, A. Loida, B. Grambow and H. Geckeis 238 (1996) 11

Ten-year results from unsaturated drip tests with UO_2 at 90°C: implications for the corrosion of spent nuclear fuel, D.J. Wronkiewicz, J.K. Bates, S.F. Wolf and E.C. Buck 238 (1996) 78

Impedance measurements on UO_{2+x} in saturated NaCl solutions, J. Engelhardt, F. Feldmaier, M. Laske and G. Marx 238 (1996) 104

Lead/carbon steel galvanic corrosion evaluation, C.J. Semino, A.L. Burkart, M.E. García and R. Cassibba 238 (1996) 198

Stress corrosion crack growth studies on AISI type 316 stainless steel in boiling acidified sodium chloride solution, T.V. Vinoy, H. Shaikh, H.S. Khatak, N. Sivaibharasi and J.B. Gnanamoorthy 238 (1996) 278

Copper, Copper Alloys and Compounds

Annihilation of freely migrating defects by cascade remnants in Cu–1% Au alloys, A. Iwase, L.E. Rehn, P.M. Baldo and L. Funk 238 (1996) 224

Effects of neutron irradiation on mechanical properties and microstructures of dispersion and precipitation hardened copper alloys, B.N. Singh, D.J. Edwards and P. Toft 238 (1996) 244

Crystallographic Properties

Analysis of the structure of layers on UO_2 leached in H_2O, Hj. Matzke 238 (1996) 58

EXAFS study of $(La_{1-x}M_x)_2Zr_2O_7$ (M = Nd and Ce), H. Yokoi, Y. Arita, T. Matsui, H. Ohno and K. Kobayashi 238 (1996) 163

Diffusion

Analysis of the structure of layers on UO_2 leached in H_2O, Hj. Matzke 238 (1996) 58

An evaluation of water layer thickness effective in the oxidation of UO_2 fuel due to radiolysis of water, H. Christensen and S. Sunder 238 (1996) 70

Modelling of short-lived fission product release behaviour during annealing conditions, B.J. Lewis, D. Evens, F.C. Iglesias and Y. Liu 238 (1996) 183
Annihilation of freely migrating defects by cascade remnants in Cu–1% Au alloys, A. Iwase, L.E. Rehn, P.M. Baldo and L. Funk 238 (1996) 224
The correspondence between self-diffusion properties and melting temperatures for α-Zr and α-Ti, N. Matsuura, G.M. Hood and H. Zou 238 (1996) 260

Experimental Techniques
Construction of an electrode for irradiated UO_2 fuel and preliminary results concerning its use in aqueous solutions, P.D.W. Bottomley, D.H. Wegen and M. Coquerelle 238 (1996) 23
Analysis of the structure of layers on UO_2 leached in H_2O, Hj. Matzke 238 (1996) 58
Impedance measurements on UO_{2+x} in saturated NaCl solutions, J. Engelhardt, F. Feldmaier, M. Laske and G. Marx 238 (1996) 104

Fatigue
Changes in mechanical properties of irradiated Zircaloy-2 fuel cladding due to short term annealing, T. Torimaru, T. Yasuda and M. Nakatsuka 238 (1996) 169

Fission Products
Leaching of SIMFUEL in simulated granitic water: comparison to results in demineralized water, V.V. Rondinella and Hj. Matzke 238 (1996) 44
Ten-year results from unsaturated drip tests with UO_2 at 90°C: implications for the corrosion of spent nuclear fuel, D.J. Wronkiewicz, J.K. Bates, S.F. Wolf and E.C. Buck 238 (1996) 78
Uraninite and UO_2 in spent nuclear fuel: a comparison, J. Janeczek, R.C. Ewing, V.M. Oversby and L.O. Werme 238 (1996) 121
The role of directed bubble diffusion to grain boundaries in post-irradiation fission gas release from UO_2: a quantitative assessment, J.H. Evans 238 (1996) 175
Modelling of short-lived fission product release behaviour during annealing conditions, B.J. Lewis, D. Evens, F.C. Iglesias and Y. Liu 238 (1996) 183
Determination of partial pressures of Te, Te_2, Ag, Ag_2, AgTe and Ag_2Te gaseous species over $Ag(s) + Ag_2Te(s)$ system, V. Venugopal, S.G. Kulkarni, A.A. Banerjee, G.A. Rama Rao, K.N. Roy and D.D. Sood 238 (1996) 218

Fuels and Fuel Elements
German approaches to closing the nuclear fuel cycle and final disposal of HLW, J.I. Kim, K. Gompper, K.D. Closs, G. Kessler and D. Faude 238 (1996) 1
Anoxic corrosion of various high burnup spent fuel samples, A. Loida, B. Grambow and H. Geckeis 238 (1996) 11
Construction of an electrode for irradiated UO_2 fuel and preliminary results concerning its use in aqueous solutions, P.D.W. Bottomley, D.H. Wegen and M. Coquerelle 238 (1996) 23
Coprecipitation phenomena during spent fuel dissolution. Part 1: Experimental procedure and initial results on trivalent ion behaviour, J. Quiñones, B. Grambow, A. Loida and H. Geckeis 238 (1996) 38
An evaluation of water layer thickness effective in the oxidation of UO_2 fuel due to radiolysis of water, H. Christensen and S. Sunder 238 (1996) 70
Uraninite and UO_2 in spent nuclear fuel: a comparison, J. Janeczek, R.C. Ewing, V.M. Oversby and L.O. Werme 238 (1996) 121
Measurements of instant-release source terms for ^{137}Cs, ^{90}Sr, ^{99}Tc, ^{129}I and ^{14}C in used CANDU fuels, S. Stroes-Gascoyne 238 (1996) 264

Gases *(excludes Hydrogen, Helium and Tritium)* **in Materials**
Modelling of short-lived fission product release behaviour during annealing conditions, B.J. Lewis, D. Evens, F.C. Iglesias and Y. Liu 238 (1996) 183

Hydrogen and Hydrides *(includes Deuterium and Deuterides)*
Compositional variation of hydrogen permeability in ferritic alloys and steels, P. Jung 238 (1996) 189

Ion Irradiation
Annihilation of freely migrating defects by cascade remnants in Cu–1% Au alloys, A. Iwase, L.E. Rehn, P.M. Baldo and L. Funk 238 (1996) 224

Kinetics
Effect of H_2O_2, NaClO and Fe on the dissolution of unirradiated UO_2 in NaCl 5 mol kg^{-1}. Comparison with spent fuel dissolution experiments, J. Giménez, E. Baraj, M.E. Torrero, I. Casas and J. de Pablo 238 (1996) 64
Precipitation kinetics of U(VI) phases formed in high concentrated saline solutions, P. Diaz Arocas 238 (1996) 96
A kinetic model for the stability of spent fuel matrix under oxic conditions, J. Bruno, E. Cera, L. Duro, T.E. Eriksen and L.O. Werme 238 (1996) 110

Mechanical Properties *(not listed elsewhere)*
A model to describe the anisotropic viscoplastic behaviour of Zircaloy-4 tubes, P. Delobelle, P. Robinet, P. Geyer and P. Bouffioux 238 (1996) 135
Changes in mechanical properties of irradiated Zircaloy-2 fuel cladding due to short term annealing, T. Torimaru, T. Yasuda and M. Nakatsuka 238 (1996) 169
Effects of alloying elements on the tensile properties and oxidation behavior of modified Zircaloy-4 in 360°C water, H.-S. Hong, S.-J. Kim and K.-S. Lee 238 (1996) 211

Metals, Alloys and Compounds *(not listed elsewhere)*
Lead/carbon steel galvanic corrosion evaluation, C.J. Semino, A.L. Burkart, M.E. García and R. Cassibba 238 (1996) 198

Neutron Irradiation
Changes in mechanical properties of irradiated Zircaloy-2 fuel cladding due to short term annealing, T. Torimaru, T. Yasuda and M. Nakatsuka 238 (1996) 169
Irradiation-induced microstructural changes in Zr–1% Sn–1% Nb–0.4% Fe, A.V. Nikulina, V.A. Markelov, M.M. Peregud, V.N. Voevodin, V.L. Panchenko and G.P. Kobylyansky 238 (1996) 205
Effects of neutron irradiation on mechanical properties and microstructures of dispersion and precipitation hardened copper alloys, B.N. Singh, D.J. Edwards and P. Toft 238 (1996) 244

Optical Microscopy
Construction of an electrode for irradiated UO_2 fuel and preliminary results concerning its use in aqueous solutions, P.D.W. Bottomley, D.H. Wegen and M. Coquerelle 238 (1996) 23

Permeation
Compositional variation of hydrogen permeability in ferritic alloys and steels, P. Jung 238 (1996) 189

Phase Transformation *(includes Evaporation, Sublimation)*
Determination of partial pressures of Te, Te_2, Ag, Ag_2, AgTe and Ag_2Te gaseous species over $Ag(s) + Ag_2Te(s)$ system, V. Venugopal, S.G. Kulkarni, A.A. Banerjee, G.A. Rama Rao, K.N. Roy and D.D. Sood 238 (1996) 218

Plutonium, Plutonium Alloys and Compounds
German approaches to closing the nuclear fuel cycle and final disposal of HLW, J.I. Kim, K. Gompper, K.D. Closs, G. Kessler and D. Faude 238 (1996) 1
Anoxic corrosion of various high burnup spent fuel samples, A. Loida, B. Grambow and H. Geckeis 238 (1996) 11

Powder Processes and Products
Sintering behaviour of niobia-doped large grain UO_2 pellet, Y. Harada 238 (1996) 237

Precipitates and Precipitation
Coprecipitation phenomena during spent fuel dissolution. Part 1: Experimental procedure and initial results on trivalent ion behaviour, J. Quiñones, B. Grambow, A. Loida and H. Geckeis 238 (1996) 38
Ten-year results from unsaturated drip tests with UO_2 at 90°C: implications for the corrosion of spent nuclear fuel, D.J. Wronkiewicz, J.K. Bates, S.F. Wolf and E.C. Buck 238 (1996) 78
Precipitation kinetics of U(VI) phases formed in high concentrated saline solutions, P. Diaz Arocas 238 (1996) 96

Radiation Effects: Atomic Defects
Annihilation of freely migrating defects by cascade remnants in Cu–1% Au alloys, A. Iwase, L.E. Rehn, P.M. Baldo and L. Funk 238 (1996) 224

Radiation Effects: Extended Defects, Microstructures
Irradiation-induced microstructural changes in Zr–1% Sn–1% Nb–0.4% Fe, A.V. Nikulina, V.A. Markelov, M.M. Peregud, V.N. Voevodin, V.L. Panchenko and G.P. Kobylyansky 238 (1996) 205
Effects of neutron irradiation on mechanical properties and microstructures of dispersion and precipitation hardened copper alloys, B.N. Singh, D.J. Edwards and P. Toft 238 (1996) 244

Radiation Effects: Mechanical Properties
Effects of neutron irradiation on mechanical properties and microstructures of dispersion and precipitation hardened copper alloys, B.N. Singh, D.J. Edwards and P. Toft 238 (1996) 244

Radiolysis
Anoxic corrosion of various high burnup spent fuel samples, A. Loida, B. Grambow and H. Geckeis 238 (1996) 11
An evaluation of water layer thickness effective in the oxidation of UO_2 fuel due to radiolysis of water, H. Christensen and S. Sunder 238 (1996) 70
A kinetic model for the stability of spent fuel matrix under oxic conditions, J. Bruno, E. Cera, L. Duro, T.E. Eriksen and L.O. Werme 238 (1996) 110

Recrystallization, Recovery and Grain Growth
Changes in mechanical properties of irradiated Zircaloy-2 fuel cladding due to short term annealing, T. Torimaru, T. Yasuda and M. Nakatsuka 238 (1996) 169

Redeposition
Leaching of SIMFUEL in simulated granitic water: comparison to results in demineralized water, V.V. Rondinella and Hj. Matzke 238 (1996) 44

Reprocessing
German approaches to closing the nuclear fuel cycle and final disposal of HLW, J.I. Kim, K. Gompper, K.D. Closs, G. Kessler and D. Faude 238 (1996) 1

Safety of Nuclear Reactors
Modelling of short-lived fission product release behaviour during annealing conditions, B.J. Lewis, D. Evens, F.C. Iglesias and Y. Liu 238 (1996) 183

Steels, Austenitic
Stress corrosion crack growth studies on AISI type 316 stainless steel in boiling acidified sodium chloride solution, T.V. Vinoy, H. Shaikh, H.S. Khatak, N. Sivaibharasi and J.B. Gnanamoorthy 238 (1996) 278

Steels, Ferritic
Compositional variation of hydrogen permeability in ferritic alloys and steels, P. Jung 238 (1996) 189
Lead/carbon steel galvanic corrosion evaluation, C.J. Semino, A.L. Burkart, M.E. García and R. Cassibba 238 (1996) 198

Surface Effects
Leaching of SIMFUEL in simulated granitic water: comparison to results in demineralized water, V.V. Rondinella and Hj. Matzke 238 (1996) 44

Theory and Modelling
The role of directed bubble diffusion to grain boundaries in post-irradiation fission gas release from UO_2: a quantitative assessment, J.H. Evans 238 (1996) 175
Modelling of short-lived fission product release behaviour during annealing conditions, B.J. Lewis, D. Evens, F.C. Iglesias and Y. Liu 238 (1996) 183

Thermophysical Properties
Determination of partial pressures of Te, Te_2, Ag, Ag_2, AgTe and Ag_2Te gaseous species over $Ag(s) + Ag_2Te(s)$ system, V. Venugopal, S.G. Kulkarni, A.A. Banerjee, G.A. Rama Rao, K.N. Roy and D.D. Sood 238 (1996) 218
The correspondence between self-diffusion properties and melting temperatures for α-Zr and α-Ti, N. Matsuura, G.M. Hood and H. Zou 238 (1996) 260

Titanium, Titanium Alloys and Compounds
The correspondence between self-diffusion properties and melting temperatures for α-Zr and α-Ti, N. Matsuura, G.M. Hood and H. Zou 238 (1996) 260

Uranium, Uranium Alloys and Compounds
Construction of an electrode for irradiated UO_2 fuel and preliminary results concerning its use in aqueous solutions, P.D.W. Bottomley, D.H. Wegen and M. Coquerelle 238 (1996) 23
Leaching of SIMFUEL in simulated granitic water: comparison to results in demineralized water, V.V. Rondinella and Hj. Matzke 238 (1996) 44
Analysis of the structure of layers on UO_2 leached in H_2O, Hj. Matzke 238 (1996) 58
Effect of H_2O_2, NaClO and Fe on the dissolution of unirradiated UO_2 in NaCl 5 mol kg^{-1}. Comparison with spent fuel dissolution experiments, J. Giménez, E. Baraj, M.E. Torrero, I. Casas and J. de Pablo 238 (1996) 64
An evaluation of water layer thickness effective in the oxidation of UO_2 fuel due to radiolysis of water, H. Christensen and S. Sunder 238 (1996) 70
Ten-year results from unsaturated drip tests with UO_2 at 90°C: implications for the corrosion of spent nuclear fuel, D.J. Wronkiewicz, J.K. Bates, S.F. Wolf and E.C. Buck 238 (1996) 78
Precipitation kinetics of U(VI) phases formed in high concentrated saline solutions, P. Diaz Arocas 238 (1996) 96
Impedance measurements on UO_{2+x} in saturated NaCl solutions, J. Engelhardt, F. Feldmaier, M. Laske and G. Marx 238 (1996) 104
A kinetic model for the stability of spent fuel matrix under oxic conditions, J. Bruno, E. Cera, L. Duro, T.E. Eriksen and L.O. Werme 238 (1996) 110
Uraninite and UO_2 in spent nuclear fuel: a comparison, J. Janeczek, R.C. Ewing, V.M. Oversby and L.O. Werme 238 (1996) 121
The role of directed bubble diffusion to grain boundaries in post-irradiation fission gas release from UO_2: a quantitative assessment, J.H. Evans 238 (1996) 175
Sintering behaviour of niobia-doped large grain UO_2 pellet, Y. Harada 238 (1996) 237
Measurements of instant-release source terms for ^{137}Cs, ^{90}Sr, ^{99}Tc, ^{129}I and ^{14}C in used CANDU fuels, S. Stroes-Gascoyne 238 (1996) 264

Wastes
German approaches to closing the nuclear fuel cycle and final disposal of HLW, J.I. Kim, K. Gompper, K.D. Closs, G. Kessler and D. Faude 238 (1996) 1
Anoxic corrosion of various high burnup spent fuel samples, A. Loida, B. Grambow and H. Geckeis 238 (1996) 11
Coprecipitation phenomena during spent fuel dissolution. Part 1: Experimental procedure and initial results on trivalent ion behaviour, J. Quiñones, B. Grambow, A. Loida and H. Geckeis 238 (1996) 38
Leaching of SIMFUEL in simulated granitic water: comparison to results in demineralized water, V.V. Rondinella and Hj. Matzke 238 (1996) 44
Effect of H_2O_2, NaClO and Fe on the dissolution of unirradiated UO_2 in NaCl 5 mol kg^{-1}. Comparison with spent fuel dissolution experiments, J. Giménez, E. Baraj, M.E. Torrero, I. Casas and J. de Pablo 238 (1996) 64
Ten-year results from unsaturated drip tests with UO_2 at 90°C: implications for the corrosion of spent nuclear fuel, D.J. Wronkiewicz, J.K. Bates, S.F. Wolf and E.C. Buck 238 (1996) 78
Precipitation kinetics of U(VI) phases formed in high concentrated saline solutions, P. Diaz Arocas 238 (1996) 96
Impedance measurements on UO_{2+x} in saturated NaCl solutions, J. Engelhardt, F. Feldmaier, M. Laske and G. Marx 238 (1996) 104
Uraninite and UO_2 in spent nuclear fuel: a comparison, J. Janeczek, R.C. Ewing, V.M. Oversby and L.O. Werme 238 (1996) 121
EXAFS study of $(La_{1-x}M_x)_2Zr_2O_7$ (M = Nd and Ce), H. Yokoi, Y. Arita, T. Matsui, H. Ohno and K. Kobayashi 238 (1996) 163
Measurements of instant-release source terms for ^{137}Cs, ^{90}Sr, ^{99}Tc, ^{129}I and ^{14}C in used CANDU fuels, S. Stroes-Gascoyne 238 (1996) 264

Zirconium, Zirconium Alloys and Compounds

A model to describe the anisotropic viscoplastic behaviour of Zircaloy-4 tubes, P. Delobelle, P. Robinet, P. Geyer and P. Bouffioux 238 (1996) 135

Changes in mechanical properties of irradiated Zircaloy-2 fuel cladding due to short term annealing, T. Torimaru, T. Yasuda and M. Nakatsuka 238 (1996) 169

Lead/carbon steel galvanic corrosion evaluation, C.J. Semino, A.L. Burkart, M.E. García and R. Cassibba 238 (1996) 198

Irradiation-induced microstructural changes in Zr–1% Sn–1% Nb–0.4% Fe, A.V. Nikulina, V.A. Markelov, M.M. Peregud, V.N. Voevodin, V.L. Panchenko and G.P. Kobylyansky 238 (1996) 205

Effects of alloying elements on the tensile properties and oxidation behavior of modified Zircaloy-4 in 360°C water, H.-S. Hong, S.-J. Kim and K.-S. Lee 238 (1996) 211

The correspondence between self-diffusion properties and melting temperatures for α-Zr and α-Ti, N. Matsuura, G.M. Hood and H. Zou 238 (1996) 260